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**Monte Carlo simulations of the pulsed thermal neutron flux
in two-region hydrogenous systems
(using standard MCNP data libraries)**

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Abstract

Monte Carlo simulations of the pulsed neutron experiment in two-region systems (two concentric spheres and two coaxial finite cylinders) are presented. The MCNP code is used. Aqueous solutions of H_3BO_3 or KCl are used in the inner region. The outer region is the moderator of Plexiglas. Standard data libraries of the thermal neutron scattering cross-sections of hydrogen in hydrogenous substances are used. The time-dependent thermal neutron transport is simulated when the inner region has a constant size and the external size of the surrounding outer region is variable. The time decay constant of the thermal neutron flux in the system is found in each simulation. The results of the simulations are compared with results of real pulsed neutron experiments on the corresponding systems.

1. Introduction

Czubek's pulsed neutron method [Czubek *et al.*, 1996] of measuring the thermal neutron macroscopic absorption cross-section Σ_a in small samples is based on a combination of a theoretical approach with experimental results. The theory is based on analytical solutions of the thermal neutron diffusion in a two-region geometry. During measurements the sample-moderator system is irradiated by a burst of 14 MeV neutrons which are slowed down in the system and the thermal neutrons escaping from the system are detected. In consecutive measurements the sample of well-defined and fixed dimensions is enveloped in shells of the moderator of variable thickness and covered with a cadmium shield. A scheme of the experimental set-up is given in Fig. 1. The time decay constant λ of the fundamental mode, $\sim e^{-\lambda t}$, of the pulsed thermal neutron flux in the system is measured. The absorption cross-section Σ_a of the material of the inner sample is interpreted from the measurement results and from the calculation of a certain theoretical curve in such two-region system performed under particular assumptions.

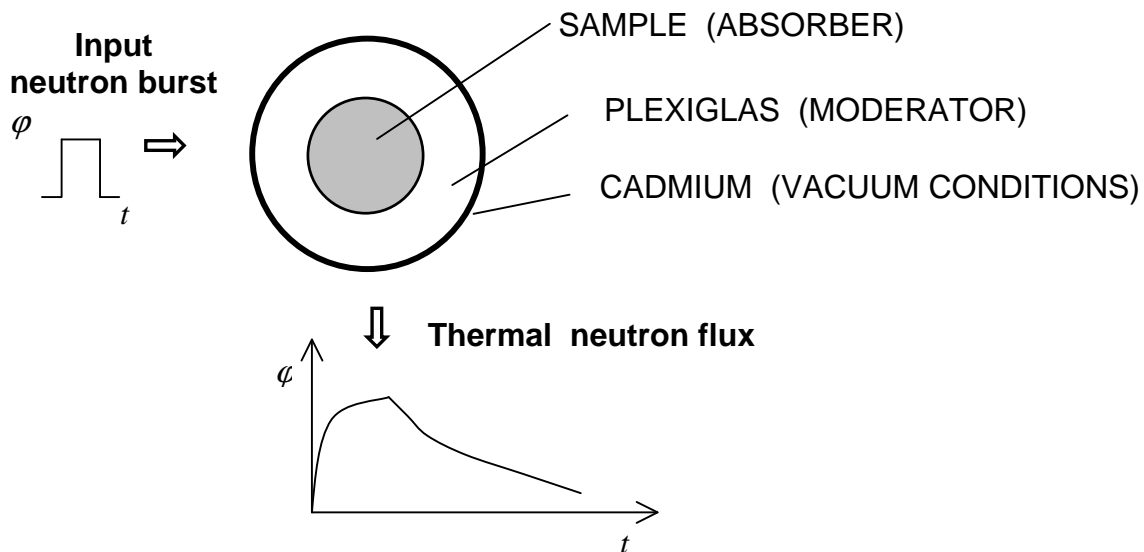


Fig.1. Scheme of the pulsed two-region system.

Materials of a very high purity and, thus, of the exactly known absorption of thermal neutrons are required to verify experimentally the method. It is very difficult to fulfil these requirements and the experimental procedure is time consuming. Computer simulations of the experiments can be very helpful or even can substitute the real measurements. The present work is aimed to compare the results of simulations of the pulsed neutron experiment in Czubek's method by the Monte Carlo method with results of the real experiment and to answer whether these simulations can help in interpretation of difficult experimental cases in

Czubek's method. The simulations have been performed in two-region systems in spherical and cylindrical geometry.

There are various large computational transport codes, as MORSE, FLUKA, MCNP, which use the Monte Carlo method. Only MCNP code allows us to simulate and investigate time-dependent neutron fields. The pulsed neutron experiments were earlier simulated for homogeneous media, as polyethylene [Dąbrowska and Drozdowicz, 2000] and for basic rock minerals [Drozdowicz *et al.*, 2003]. Now, the computer simulations have been performed of the pulsed neutron experiments in two-region geometry like that used in Czubek's method.

2. Hydrogenous moderator

Plexiglas is used in Czubek's method as the outer moderator to obtain a sufficient number of thermal neutrons, which create the thermal neutron field in the investigated system. Plexiglas [poly(methyl methacrylate), $(C_5H_8O_2)_n$] contains hydrogen atoms bound in the molecular system. For most elements the energy-dependent scattering cross-section $\sigma_s(E)$ for thermal neutrons is well described by formula of the free gas model, even for bound atoms. Moreover, it can be often assumed with a sufficient accuracy as a constant, $\sigma_s(E) \approx \sigma_{sf}$, where σ_{sf} is the free atom scattering cross-section. However, this is not the case of a hydrogenous medium because neutron scattering on hydrogen nuclei bound in a molecule is affected by the molecular dynamics, characterised with internal energy modes which are comparable with the incident neutron energies. The scattering cross-section $\sigma_s(E)$ and the average cosine of the scattering angle $\mu(E)$ are then strongly dependent on the incident neutron energy and these dependences are different for different compounds. A single general exact relationship, valid for the energy dependence of the microscopic scattering cross-section $\sigma_s^H(E)$ of hydrogen in various chemical bindings, cannot exist. The individual energy dependence $\sigma_s^H(E)$ for a particular molecule should be used.

The pulsed neutron experiment has been simulated by the Monte Carlo method using the MCNP code in which a special procedure is present for transport of thermal neutrons through media containing hydrogen. The standard nuclear data libraries containing the hydrogen scattering cross-sections for neutrons in the thermal energy region exist for a few hydrogenous materials, such as: light and heavy water, methane, benzene, zirconium hydride and polyethylene. Structures of these molecules are simple: H_2O , D_2O , CH_4 , C_6H_6 , ZrH_2 , $(CH_2)_n$, respectively. The Plexiglas macromolecule is more complex. For neutron scattering it contains two types of the hydrogen atoms in the structural formula, considering their neighbourhood. Namely, hydrogen is present in the CH_2 and CH_3 groups. There is no data library for the MCNP code, dedicated especially for hydrogen in Plexiglas.

A procedure which can be adapted to handle by the MCNP the hydrogenous system in Plexiglas is to use the same microscopic data as those for polyethylene, $(\text{CH}_2)_n$. This procedure finds the recommendation in comparison of the experimental values of the microscopic scattering cross-sections of hydrogen in Plexiglas obtained by Drozdowicz (1989) and Sibona *et al.* (1991) with those measured in polyethylene by Granada *et al.* (1987). The energy dependence of the total scattering cross-section of hydrogen, $\sigma_s^{\text{H}}(E)$, in Plexiglas and polyethylene is similar although non-identical. Moreover, greater differences can exist between the zero- and first-order energy transfer scattering kernels, $\sigma_0(E \rightarrow E')$ and $\sigma_1(E \rightarrow E')$, in Plexiglas and polyethylene, which influence the average cosine of the scattering angle and the transport cross-section. In spite of this it is interesting to compare the λ_{MCNP} values from the MCNP-simulated experiments with the λ_{exp} values measured in real experiments. The results of the comparison are presented in the paper.

3. Simulated pulsed-neutron experiment

As said, the Monte Carlo method was found very helpful for simulations of the pulsed neutron experiments [Dąbrowska and Drozdowicz, 2000; Woźnicka *et al.*, 2000]. The MCNP 4C code [Briesmeister, 2000] has been used here for such simulations.

Neutrons in hydrogenous materials slow down very quickly and they achieve in a short time the Maxwellian energy distribution inside the system. Considering the neutron source, the test was made using once 14 MeV and, second, the Maxwellian distributed thermal neutrons. The results show no influence of the pulsed source type on the determined fundamental decay constant. Therefore, it is possible to use the Maxwellian source in simulations [Dąbrowska and Drozdowicz, 2000; Woźnicka *et al.*, 2000], which makes the Monte Carlo calculation significantly faster. Thus, the initial energies of neutrons has been sampled from the Maxwellian source at the room temperature ($E_T = kT = 0.0253$ eV). The thermal neutron source has been isotropic and the neutrons have been uniformly generated inside the sample. The initial position of neutrons has been sampled from the probability density function $p(r) \sim r^2$ for r being inside the entire system. Neutrons have been generated within a 100 μs in width neutron burst. After the neutron burst, the neutron flux in the investigated volume has been scored in 1000 time channels. The width of the channels has been adjusted for each sample individually, depending on the decay rate. The number of histories in the MCNP runs has been chosen for each sample individually to assure the error corresponding to the counting statistics in channels not worse than 0.5%.

After the neutron burst, the flux vanishes in time. Usually it can be recognised as a sum of a number of exponentials and background. We are interested in the decay constant λ of the

fundamental mode, $e^{-\lambda t}$. In order to determine the time decay constant for a given sample, the thermal neutron flux has been scored in time intervals after the source pulse. The decay constant λ of the fundamental mode thermal neutron flux for each sample has been calculated like in a real pulsed experiment [Drozdowicz *et al.*, 1993], including a procedure of the observation of behaviour of the determined value as a function of the delay time.

The relative deviation ε has been defined for comparing the time decay constants from the simulated λ_{MCNP} and real λ_{exp} experiments:

$$\varepsilon [\%] = \frac{\lambda_{\text{MCNP}} - \lambda_{\text{exp}}}{\lambda_{\text{exp}}} \cdot 100 \% \quad . \quad (3.1)$$

Simulations corresponding to the real experiments in spherical [Czubek *et al.*, 1980] and cylindrical [Krynicka *et al.*, 2000a; Krynicka *et al.*, 2001] geometry have been performed. Two kinds of samples have been used: aqueous solutions of boric acid, H_3BO_3 , and of potassium chloride, KCl. Their concentrations have been varied to vary the thermal neutron absorption cross-section.

4. Simulations in two-region spherical geometry

The simulations have been performed for the two-region spherical geometry (concentric spheres). The sample of boric acid in light water ($\text{H}_3\text{BO}_3 + \text{H}_2\text{O}$), of the known mass concentration C and the resulting known density ρ , has been surrounded by Plexiglas of density $\rho = 1.178 \text{ g/cm}^3$. The radius of the samples has been fixed $R_1 = 5.0 \text{ cm}$. The range of the outer radius of Plexiglas has been $R_{2g} = (6.5 \div 9.49) \text{ cm}$.

The proper selection of MCNP data libraries is an important procedure. Scattering cross-sections for oxygen, boron and carbon have been taken from the ENDF/B-VI.0 standard MCNP data library. For scattering of thermal neutrons on hydrogen the $S(\alpha, \beta)$ model has been used:

- H in H_2O : TMCCS1, lwtr.01t,
- H in H_3BO_3 is treated as in water,
- H in Plexiglas: TMCCS1, poly.01t (which is for H in polyethylene).

Boron is an element characterising with a high absorption of thermal neutrons. The abundance of the contributing isotopes, ^{11}B and ^{10}B , is very important because their microscopic absorption cross-sections differ extremely. The natural boron isotopic abundance fluctuates slightly [Rosman and Taylor, 1998], which can lead to an uncertainty of the microscopic absorption cross-section σ_{aB} . In this calculations it has been assumed as 20% ^{10}B and 80% ^{11}B according to Mughabghab *et al.* (1981).

The obtained time decay constants of the neutron flux λ_{MCNP} has been compared with the λ_{exp} values known from experiments [Czubek *et al.*, 1980]. The results are listed in Table 1. The standard deviations $\sigma(\lambda)$ of the λ_{exp} and λ_{MCNP} decay constants have been obtained from the variance.

Table 1. Time decay constants λ from the simulated and real experiments in two-region spherical geometry. The H_3BO_3 reference solutions in the inner region.

C	ρ	R_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ε
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
1.75	1.00475	6.5	35 226	33 272	5.87
			24	486	5.75
		7.5	24 468	23 357	4.76
			24	163	4.31
0.875	1.0017	6.5	24 777	24 776	0.00
			51	112	0.02
		6.99	22 210	21 286	4.34
			32	192	4.32
		7.5	19 581	18 893	3.64
			20	84	4.15
		9.49	12 508	11 819	5.83
			4	182	5.74
			246		

The average relative deviation ε equals 4.06 %.

The plot λ_{MCNP} vs λ_{exp} is shown in Fig. 2. If the results of the simulations and experiments were in a perfect agreement they would lay on the diagonal.

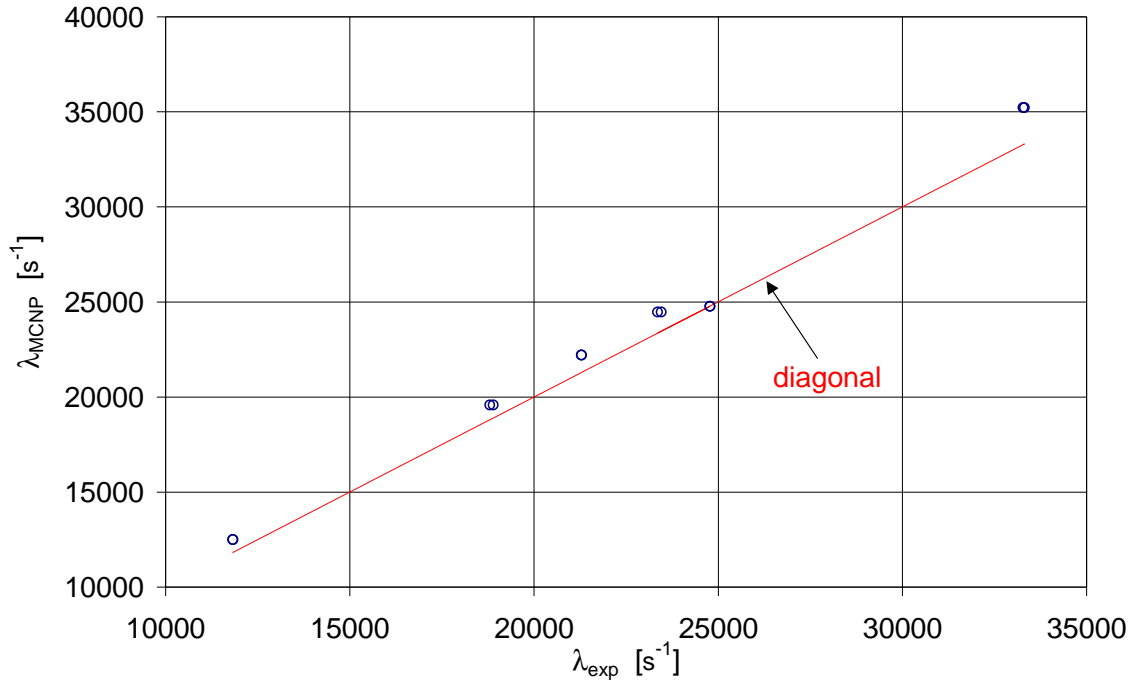


Fig. 2. Decay constant λ_{MCNP} vs λ_{exp} in two-region spherical geometry with H_3BO_3 reference solutions in the inner region.

5. Simulations in the two-region cylindrical geometry

The second series of simulations has been performed for the two-region cylindrical geometry (coaxial cylinders). The samples of boric acid in light water ($\text{H}_3\text{BO}_3 + \text{H}_2\text{O}$) and potassium chloride in light water ($\text{KCl} + \text{H}_2\text{O}$) of the known mass concentrations C and the resulting densities ρ , have been surrounded by Plexiglas of the variable size ($H_{2g} = 2R_{2g}$, where H_{2g} and R_{2g} are the external height and radius of the cylinder). The Plexiglas density has been equal $\rho = 1.1764 \text{ g/cm}^3$. The size of the samples has been constant ($H_1 = 2R_1 = 6.0 \text{ cm}$) and the range of the outer cylinder radius has been $R_{2g} = (4.5 \div 7.0) \text{ cm}$.

Scattering cross-sections for oxygen, boron, potassium, chlorine and carbon have been taken from the ENDF/B-VI.0 standard MCNP data library. For scattering of thermal neutrons on hydrogen the $S(\alpha, \beta)$ model has been used:

- H in H_2O : TMCCS1, lwtr.01t,
- H in H_3BO_3 is treated as in water,
- H in Plexiglas: TMCCS1, poly.01t (which is for H in polyethylene).

The abundances of ^{10}B and ^{11}B for the H_3BO_3 material used in experiments were found in independent measurements [Krynicka *et al.*, 2000b]. They are 19.55% ^{10}B and 80.45% ^{11}B .

The obtained time decay constants of the neutron flux λ_{MCNP} have been compared with the λ_{exp} values known from the experiments [Krynicka *et al.*, 2000a; Krynicka *et al.*, 2001]. The results are listed in Tables 2 and 3. The plot λ_{MCNP} vs λ_{exp} is shown in Fig. 3 and Fig. 4.

Table 2. Time decay constants λ from the simulated and real experiments in two-region cylindrical geometry. The H_3BO_3 reference solutions in the inner region.

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ		
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]		
1.0000	1.0022	10.4	22 620 49	22 037	2.65		
				122	2.40		
				22 090 184	2.34		
				22 102 81	2.80		
		10.8	21 125 37	20 722	1.94		
				144	2.32		
				20 647 97	2.42		
				20 625 68	2.14		
		11.2	19 817 36	20 683	2.14		
				120	2.39		
				19 355	2.39		
				65	2.68		
					19 299	88	2.32
					19 367	71	2.35
					19 362	91	2.05
		1.1000	1.0025	10.0	24 887 70	24 386	2.72
105	2.43						
24 229 80	2.51						
24 297 83	2.01						
10.4	23 181 42			22 724	2.01		
				111	2.38		
				22 643 119	2.43		
				22 630 154			

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
1.2000	1.0029	10.8	21 624 42	22 693	2.15
				89	2.14
				21 170 73	2.57
				21 083 84	2.73
		10.0	25 506 60	25 005	2.00
				86	2.54
				24 875 112	2.25
				24 945 116	2.25
		10.4	23 724 48	24 944	2.25
				96	2.01
				23 256	2.45
				145	2.16
			23 157	173	2.22
			23 222	114	2.20
			23 208	127	2.21
			21 630	89	1.37
1.4010	1.0036	9.6	28 843 70	21 628	1.76
				85	1.81
				21 808 113	1.94
				21 724 88	1.55
		10.8	22 106 54	28 329	1.81
				120	1.94
				28 293 81	1.55
				28 403 139	2.20
			28 223	149	

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ	
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]	
		10.0	26 706 65	26 177	2.02	
				104	2.04	
				26 172	2.57	
				81	2.35	
		10.4	24 710 40	24 235	102	1.96
					24 251	1.89
					198	1.82
					24 269	2.34
		9.0	34 448 72	33 413	255	3.10
					33 299	3.45
					319	3.27
					33 356	3.32
9.6	30 322 75	29 662	164	2.23		
			29 561	2.57		
			162	2.24		
			29 657	2.74		
10.0	27 934 51	27 216	174	2.64		
			27 196	2.71		
			226	2.79		
			27 177	2.74		
1.6011	1.0042	9.0	34 448 72	33 413	3.10	
				255	3.45	
1.8000	1.0049	9.0	35 928 56	34 696	3.55	
				614	3.88	

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ	
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]	
		9.6	31 420 57	34 692	3.56	
				517	3.59	
				34 684	2.43	
				371	2.14	
		10.0	28 856 37	28 155	109	2.21
					30 762	2.33
					145	2.49
					30 742	2.34
		10.0	28 856 37	28 155	109	2.52
					28 197	2.58
					150	2.10
					28 147	2.72
8.2	44 942 78	44 019	310	2.83		
			43 754	2.89		
			246	3.11		
			43 705	3.19		
8.6	41 063 136	39 823	306	2.94		
			39 792	1.61		
			173	1.91		
			39 889	1.71		
9.0	37 407 92	36 705	120	1.91		
			594	1.71		
			40 411	1.91		
			221	1.71		
2.0014	1.0056	8.2	44 942 78	44 019	2.10	
				310	2.72	
		8.6	41 063 136	39 823	3.11	
				306	3.19	
		9.0	37 407 92	36 705	1.91	
				120	1.71	

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
2.1000	1.0060	8.2	45 963 185	45 174	1.75
				351	
				44 987	2.17
				206	
			45 065	1.99	
			368		
			45 180	1.73	
			262		
		8.6	41 904 151	41 054	2.07
				451	
				40 920	2.40
				268	
		41 150	339	41 150	1.83
				339	
				40 861	2.55
				147	
9.0	37 998 49	37 523	1.27		
		188			
		37 350	1.73		
		163			
37 512	1.30				
197					
37 539	1.22				
332					
2.2000	1.0063	8.2	47 217 160	46 202	2.20
				380	
				46 145	2.32
				236	
			45 902	2.86	
			382		
			45 985	2.68	
			361		
		8.6	42 868 107	41 413	3.51
				250	
				41 480	3.35
				383	
		41 318	434	41 318	3.75
				434	
				41 989	2.09
				169	
9.0	38 234 42	37 831	1.07		
		284			
		37 876	0.95		
		211			

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
2.3000	1.0067	8.2	48 179 156	37 910	0.85
				328	
				37 842	1.04
				235	
				46 901	2.72
				234	
				47 184	2.11
				219	
		47 142	338	47 142	2.20
				338	
				47 113	2.26
				263	
		8.6	43 717 141	42 572	2.69
				216	
				42 131	3.76
				260	
42 602	2.62				
199					
42 652	2.50				
290					
9.0	39 445 51	38 727	1.85		
		143			
		38 572	2.26		
		131			
2.4000	1.0070	8.2	49 037 100	47 823	2.54
				239	
				48 184	1.77
				397	
				47 785	2.62
				469	
				48 348	1.43
				386	
8.6	44 456 134	43 143	3.04		
		162			
		43 258	2.77		
		217			
		42 371	4.92		
		221			
		43 212	2.88		
		285			
42 642	4.25				
332					
42 766	3.95				
523					

Table 2. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
		9.0	40 035 49	38 701 291	3.45
			39 032 274	2.57	
			38 569 520	3.80	
			38 923 245	2.86	
			38 827 136	3.11	
			38 920 342	2.86	

The average relative deviation ϵ equals 2.44 %.

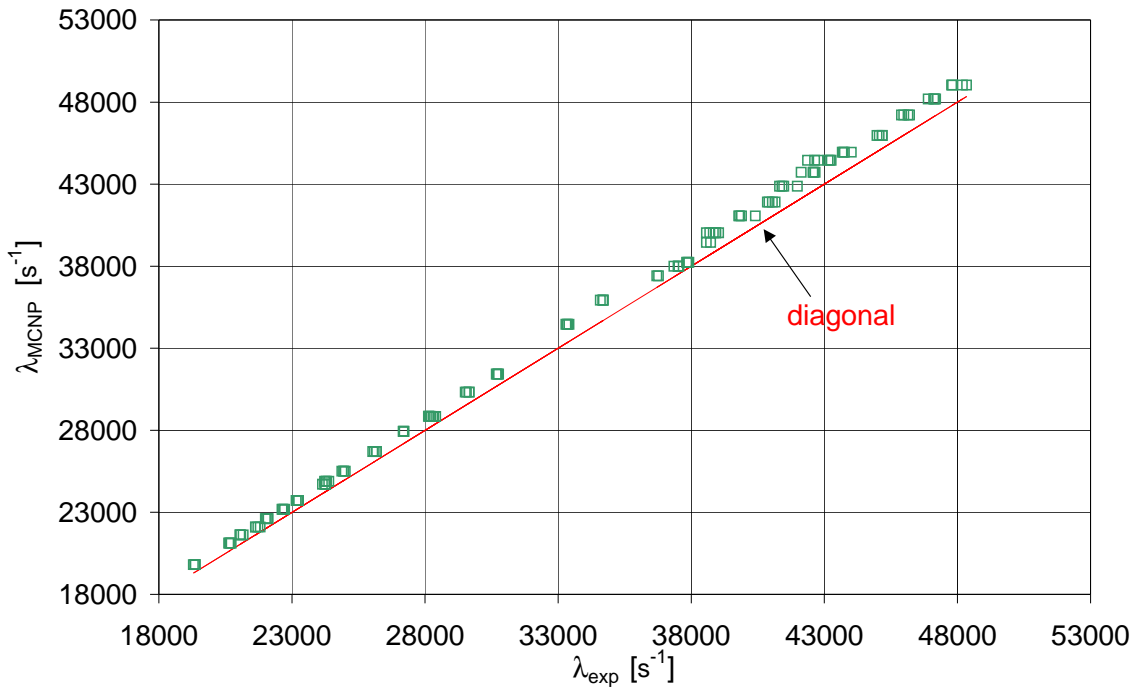


Fig. 3. Decay constant λ_{MCNP} vs λ_{exp} in two-region cylindrical geometry with the H_3BO_3 reference solutions in the inner region.

Table 3. Time decay constants λ from the simulated and real experiments in two-region cylindrical geometry. The KCl reference solutions in the inner region.

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ	
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]	
11.500	1.0734	12.8	13 754 29	13 482	2.02	
				23	1.93	
			13 494 22	1.93		
				2.06		
			13.2	13 153 26	12 887	2.06
					42	2.18
		12 872 103	2.18			
			2.02			
		13.6	12 602 13	12 399	1.64	
				60	2.02	
		12 352 38	2.02			
			1.82			
14.0	12 114 18	11 898	1.82			
		50	1.93			
11 885 51	1.93					
	1.98					
13.000	1.0836	12.4	14 708 20	14 423	1.98	
				75	2.07	
			14 410 46	2.07		
				1.94		
			14 428 71	1.94		
				1.93		
		12.8	14 022 18	13 757	1.93	
				86	1.96	
		13 752 46	1.96			
			2.16			
		13 725 52	2.16			
			2.02			
13.2	13 411 25	13 145	2.02			
		84	2.03			
13 144 62	2.03					
	1.81					
13 173 62	1.81					
	2.14					
13.6	12 859 24	12 590	2.14			
		39	2.07			
12 598 63	2.07					

Table 3. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
15.000	1.0974	11.6	16 757 26	12 574	2.27
				78	
				16 474	1.72
				49	
				16 453	1.85
				39	
				16 416	2.08
				56	
				16 534	1.35
				85	
				16 462	1.79
				97	
		16 534	1.35		
		94			
		12.0	15 866 22	15 595	1.74
				67	
				15 578	1.85
				44	
15 595	1.74				
59					
15 583	1.82				
19					
12.4	15 109 27	14 764	2.34		
		51			
		14 874	1.58		
		66			
14 760	2.36				
84					
14 889	1.48				
97					
12.8	14 399 30	14 125	1.94		
		41			
		14 104	2.09		
		114			
14 077	2.29				
50					
14 162	1.67				
53					

Table 3. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
18.000	1.01185	10.8	19 622 29	19 370	1.30
				19 204 37	2.18
		11.2	18 499 28	18 258	1.32
				18 254 53	1.34
				18 157 86	1.88
				18 225 128	1.50
		11.6	17 462 22	17 137	1.90
				17 185 62	1.61
				17 183 94	1.62
				17 160 59	1.76
		12.0	16 524 22	16 225	1.84
				16 232 35	1.80
				16 213 76	1.92
				16 216 32	1.90
		12.4	15 668 13	15 351	2.07
				15 344 48	2.11
21.000	1.1401	10.4	21 890 42	21 544	1.61
				21 532 67	1.66
				21 547 117	1.59
				21 538 111	1.63

Table 3. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ϵ		
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]		
		10.8	20 522 41	20 095	2.12		
				20 176 90	1.71		
				20 157 77	1.81		
				20 152 89	1.84		
				11.2	19 263 21	18 944	1.68
				18 924 77	1.79		
		11.6	18 154 30	18 883	2.01		
				18 893 66	1.96		
				17 797	2.01		
				17 755 100	2.25		
		23.600	1.1593	10.0	24 324 44	17 812	1.92
						17 859 57	1.65
						23 972	1.47
						24 079 83	1.02
		10.4	22 663 26	22 259	22 259	1.81	
					22 334 116	1.47	
22 305 69	1.61						
22 249 104	1.86						

Table 3. (continued)

C	ρ	H_{2g}	λ_{MCNP} $\sigma(\lambda)$	λ_{exp} $\sigma(\lambda)$	ε
[%]	[g/cm ³]	[cm]	[s ⁻¹]	[s ⁻¹]	[%]
		10.8	21 207 29	20 791 76	2.00
				20 787 76	2.02
				20 774 160	2.08
				20 827 71	1.82
		11.2	19 900 32	19 490 37	2.10
				19 512 62	1.99

The average relative deviation ε equals 1.84%.

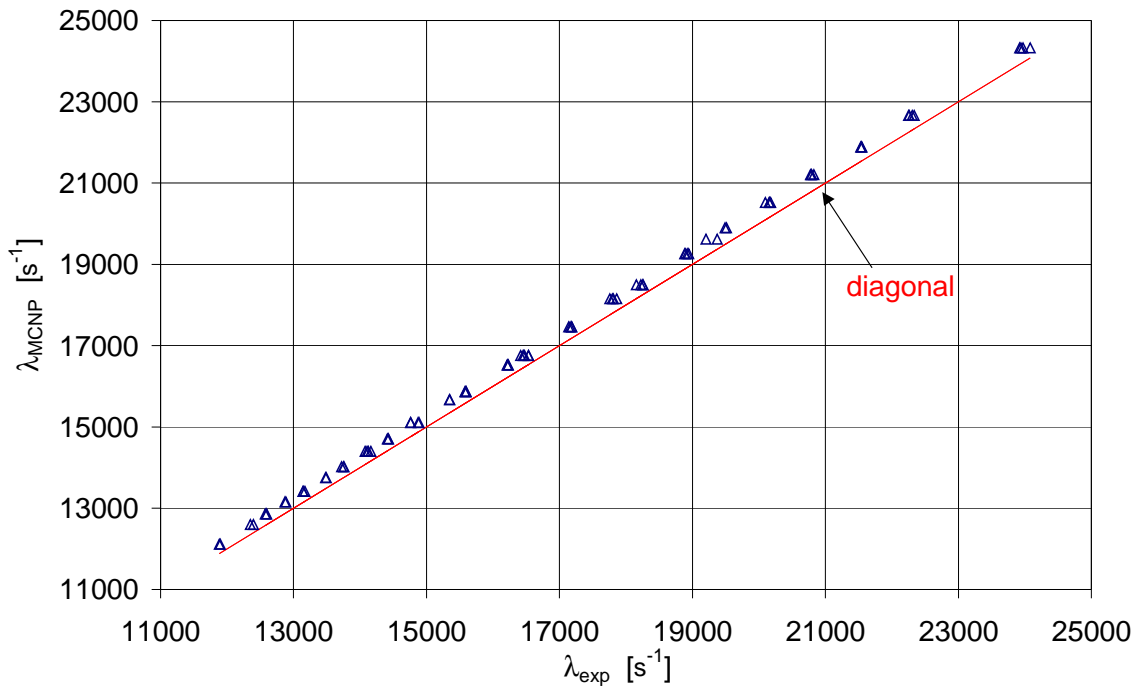


Fig. 4. Decay constant λ_{MCNP} vs λ_{exp} in two-region cylindrical geometry with the KCl reference solutions in the inner region.

The useful range of the time decay constants λ in Czubek's method covers the interval $12\,000\text{ s}^{-1} \leq \lambda \leq 50\,000\text{ s}^{-1}$. It can be achieved when both reference materials (the aqueous solutions of H_3BO_3 and KCl) are taken into account. The time decay constants λ_{MCNP} vs λ_{exp} for both materials together are plotted in Fig.5. The average relative deviation ε equals 2.20%.

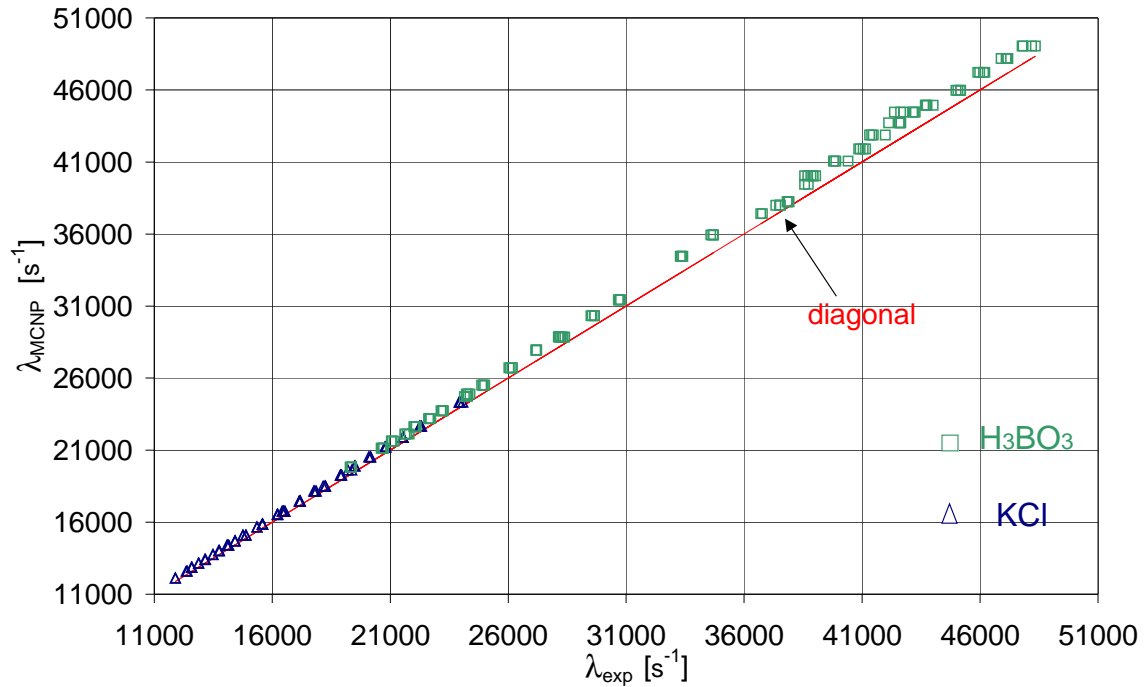


Fig. 5. Decay constant λ_{MCNP} vs λ_{exp} in two-region cylindrical geometry (H_3BO_3 or KCl reference solutions in the inner region).

6. Conclusions

The performed calculations have allowed us to obtain the routine way of computer simulations of the pulsed neutron experiments in two-region geometry like that used in Czubek's method. The fundamental decay constants λ_{MCNP} of the thermal neutron flux have been compared with the λ_{exp} values measured in real experiments.

A few experiments made in the overlapping intervals of the absorption resulting from the H_3BO_3 and KCl solutions show a very good conformity of the results. In all cases the λ_{MCNP} values are greater than the λ_{exp} values. The maximum relative deviation ε between these values has been equal to 5.87%. The average deviation in the two-region cylindrical geometry has been ca. 2%. The most probable reason of this behaviour comes from the use in the simulations the scattering library of hydrogen in polyethylene recommended for hydrogen in Plexiglas (as there is no separate library for the latter material). An improvement of this library is necessary for more precise quantitative calculations.

References

- Briesmeister J. F. (2000)
Monte Carlo N-Particle Transport Code System. Version 4C.
LA-13709-M, Los Alamos National Laboratory.
- Czubek J. A., Drozdowicz K., Gabańska B., Igielski A., Krynicka E., Woźnicka U. (1996)
Thermal neutron macroscopic absorption cross-section measurement applied for geophysics.
Progr. Nucl. Energy **30**, 295-303.
- Czubek J. A., Drozdowicz K., Igielski A., Krynicka-Drozdowicz E., Sobczyński Z., Woźnicka U. (1980)
Thermal neutron absorption cross-section for small samples (Experiments in spherical geometry).
Rept. INP No. 1119/AP, Institute of Nuclear Physics, Kraków.
- Dąbrowska J., Drozdowicz K. (2000)
Monte Carlo simulation of pulsed neutron experiments on samples of variable mass density.
Nucl. Instrum. Meth. A **443**, 416-421.
- Drozdowicz K. (1989)
Total cross-section of Plexiglass in the thermal neutron region.
Ann. Nucl. Energy **16**, 275-278.
- Drozdowicz K., Gabańska B., Krynicka E. (1993)
Fitting the decaying experimental curve by sum of exponentials.
Rept. INP No. 1635/PM, Institute of Nuclear Physics, Kraków.
- Drozdowicz K., Krynicka E., Dąbrowska J. (2003)
Diffusion cooling of thermal neutrons in basic rock minerals by Monte Carlo simulation of the pulsed neutron experiments.
Appl. Radiat. Isot. **58**, 727-733.
- Granada J. R., Dawidowski J., Mayer R. E., Gillette V. H. (1987)
Thermal neutron cross-section and transport properties in polyethylene.
Nucl. Instrum. Meth. A **261**, 573-578.
- Krynicka E., Drozdowicz K., Gabańska B., Janik W., Dąbrowska J. (2000a)
Measurements of the thermal neutron absorption Σ_a of model systems (reference KCl solutions) in two-region geometry.
Rept. INP No. 1857/PN, Institute of Nuclear Physics, Kraków.
<http://www.ifj.edu.pl/reports/2000>.
- Krynicka E., Gabańska B., Woźnicka U., Dąbrowska J., Burda J. (2001)
Interpretation problems of Czubek's Σ_a measurements method. Reference experiment on H_3BO_3 solutions.
Rept. INP No. 1890/PN, Institute of Nuclear Physics, Kraków.
<http://www.ifj.edu.pl/reports/2001>.
- Krynicka E., Gabańska B., Woźnicka U., Kosik M. (2000b)
Measurements of the thermal neutron absorption Σ_a of boron of unknown isotopic ratio.
Rept. INP No. 1860/PN, Institute of Nuclear Physics, Kraków.
<http://www.ifj.edu.pl/reports/2000>.

- Mughabghab S. F., Divadeenam M., Holden W. E. (1981)
Neutron cross-sections, vol.1. Neutron resonance parameters and thermal cross-sections
Part A: Z = 1 – 60.
Academic Press, New York.
- Rosman K. J. R., Taylor P. D. P. (1998)
Isotopic compositions of the elements 1997.
Pure & Appl. Chem. **70**, 217-235.
- Sibona G., Mayer R. E., Gillette V. H., Bonetto C., Granada J. R. (1991)
Thermal neutron cross-sections and diffusion parameters of Plexiglass.
Ann. Nucl. Energy **18**, 689-696.
- Woźnicka U., Drozdowicz K., Dąbrowska J. (2000)
A generalised interpretation of buckling experiments for thermal neutrons.
Nucl. Instrum. Meth. A **455**, 660-669.